



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**

REGION III  
2443 WARRENVILLE ROAD, SUITE 210  
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January 21, 2009

Mr. Michael W. Rencheck  
Senior Vice President and  
Chief Nuclear Officer  
Indiana Michigan Power Company  
Nuclear Generation Group  
One Cook Place  
Bridgman, MI 49106

SUBJECT: D. C. COOK NUCLEAR POWER PLANT, UNITS 1 AND 2 INTEGRATED  
INSPECTION REPORT; 05000315/2008005; 05000316/2008005

Dear Mr. Rencheck:

On December 31, 2008, the U. S. Nuclear Regulatory Commission (NRC) completed an inspection at your D. C. Cook Nuclear Power Plant, Units 1 and 2. The enclosed report documents the inspection results, which were discussed on January 8, 2009, with Mr. L. Weber and other members of your staff.

This inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

**/RA/**

Jamnes L. Cameron, Chief  
Projects Branch 6  
Division of Reactor Projects

Docket Nos. 50-315; 50-316  
License Nos. DPR-58; DPR-74

Enclosure: Inspection Report No. 05000315/2008005; 05000316/2008005  
w/Attachment: Supplemental Information

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Letter to M. Rencheck from J. Cameron dated January 21, 2009

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cc w/encl: L. Weber, Site Vice President  
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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-315; 50-316  
License Nos: DPR-58; DPR-74

Report Nos. 05000315/2008005; 05000316/2008005

Licensee: Indiana Michigan Power Company

Facility: D. C. Cook Nuclear Power Plant, Units 1 and 2

Location: Bridgman, MI

Dates: October 1 thru December 31, 2008

Inspectors: J. Lennartz, Senior Resident Inspector  
P. LaFlamme, Resident Inspector  
R. Jickling, Senior Emergency Preparedness Inspector  
F. Tran, Reactor Engineer  
R. Winter, Reactor Inspector

Approved by: Jamnes L. Cameron, Chief  
Branch 6  
Division of Reactor Projects

Enclosure

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## SUMMARY OF FINDINGS

IR 05000315/2008005; 05000316/2008005; 10/01/2008 – 12/31/2008; D.C. Cook Nuclear Power Plant, Units 1 & 2; Routine Integrated Inspection Report.

This report covers a three-month period of inspection by resident inspectors and announced baseline inspections by regional inspectors. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December 2006.

**A. NRC-Identified and Self-Revealed Findings**

No findings of significance were identified.

**B. Licensee-Identified Violations**

No violations of significance were identified.

## REPORT DETAILS

### Summary of Plant Status

Unit 1 remained in Mode 5, Cold Shutdown, during the entire inspection period.

Unit 2 operated at or near full power during the entire inspection period.

#### 1R01 Adverse Weather Protection (71111.01)

##### .1 Winter Seasonal Readiness Preparations

###### a. Inspection Scope

The inspectors conducted a review of the licensee's preparations for winter conditions to verify that the plant's design features and implementation of procedures were sufficient to protect mitigating systems from the effects of adverse weather. Documentation for selected risk-significant systems was reviewed to ensure that these systems would remain functional when challenged by inclement weather. During the inspection, the inspectors focused on plant specific design features and the licensee's procedures used to mitigate or respond to adverse weather conditions. Additionally, the inspectors reviewed the Updated Final Safety Analysis Report (UFSAR) and performance requirements for systems selected for inspection, and verified that operator actions were appropriate as specified by plant specific procedures. Cold weather protection, such as heat tracing and area heaters, was verified to be in operation where applicable. The inspectors also reviewed corrective action program (CAP) items to verify that the licensee was identifying adverse weather issues at an appropriate threshold and entering them into their CAP in accordance with station corrective action procedures. Specific documents reviewed during this inspection are listed in the Attachment. The inspectors' reviews focused specifically on the following plant systems due to their risk significance or susceptibility to cold weather issues:

- Unit 1 and Unit 2 Refueling Water Storage Tank Systems
- North and South Fire Protection Storage Tank and Piping Systems
- Unit 1 and Unit 2 East and West Main Steam Enclosures

This inspection constituted one winter seasonal readiness preparations sample as defined in IP 71111.01-05.

###### b. Findings

No findings of significance were identified.

##### .2 Readiness For Impending Adverse Weather Condition – Winter Storm Warning

###### a. Inspection Scope

On Monday November 17, 2008, a winter storm warning was issued for high winds and heavy snow squalls in the vicinity of the plant. The inspectors observed the licensee's preparations and planning for the predicted significant winter weather. The inspectors reviewed licensee procedures and discussed potential compensatory measures with

control room personnel. The inspectors focused on plant management's actions for implementing the station's procedures for ensuring adequate personnel for safe plant operation and emergency response would be available. The inspectors conducted a site walkdown including walkdowns of various plant structures and systems to check for maintenance or other apparent deficiencies that could affect system operations during the predicted significant weather. The inspectors' reviews focused specifically on the following plant systems due to their risk significance or susceptibility to the impending winter weather:

- Fire Protection Pump House
- Screen House Area Ventilation System
- Supplemental Diesel Generator and Switchgear Enclosures

The inspectors also reviewed CAP items to verify that the licensee was identifying adverse weather issues at an appropriate threshold and entering them into their CAP in accordance with station corrective action procedures. Specific documents reviewed during this inspection are listed in the Attachment.

This inspection constituted one readiness for impending adverse weather condition sample as defined in IP 71111.01-05.

b. Findings

No findings of significance were identified.

1R04 Equipment Alignment (71111.04)

a. Inspection Scope

The inspectors performed partial system walkdowns of the following risk-significant systems:

- Unit 1 available equipment options for core cooling strategy
- Unit 2 East Residual Heat Removal Train
- Unit 1 CD Emergency Diesel Generator

The inspectors selected these systems based on their risk significance relative to the reactor safety cornerstones at the time they were inspected. The inspectors attempted to identify any discrepancies that could impact the function of the system, and, therefore, potentially increase risk. The inspectors reviewed applicable operating procedures, system diagrams, UFSAR, TS requirements, outstanding work orders, condition reports, and the impact of ongoing work activities on redundant trains of equipment in order to identify conditions that could have rendered the systems incapable of performing their intended functions. The inspectors also walked down accessible portions of the systems to verify system components and support equipment were aligned correctly and operable. The inspectors examined the material condition of the components and observed operating parameters of equipment to verify that there were no obvious deficiencies. The inspectors also verified that the licensee had properly identified and resolved equipment alignment problems that could cause initiating events or impact the capability of mitigating systems or barriers and entered them into the CAP with the

appropriate significance characterization. Documents reviewed are listed in the Attachment.

These activities constituted three partial system walkdown samples as defined in IP 71111.04-05.

b. Findings

No findings of significance were identified.

1R05 Fire Protection (71111.05)

a. Inspection Scope

The inspectors conducted fire protection walkdowns which were focused on availability, accessibility, and the condition of firefighting equipment in the following risk-significant plant areas:

- Fire Zone 1E and 1F, Unit 2 East and West Containment Spray Pump Rooms
- Fire Zone 16, Unit 1 AB Emergency Diesel Generator Room
- Fire Zone 17C, Unit 1/2 Corridor to Auxiliary Feed Pump Rooms
- Fire Zone 50, Unit 2 Heating Ventilation Air Conditioning Vestibule
- Fire Zone 17B, Unit 2 West Motor Driven Auxiliary Feed Pump Room

The inspectors reviewed areas to assess if the licensee had implemented a fire protection program that adequately controlled combustibles and ignition sources within the plant, effectively maintained fire detection and suppression capability, maintained passive fire protection features in good material condition, and had implemented adequate compensatory measures for out of service, degraded or inoperable fire protection equipment, systems, or features in accordance with the licensee's fire plan. The inspectors selected fire areas based on their overall contribution to internal fire risk as documented in the plant's Individual Plant Examination of External Events with later additional insights, their potential to impact equipment which could initiate or mitigate a plant transient, or their impact on the plant's ability to respond to a security event. Using the documents listed in the attachment, the inspectors verified that fire hoses and extinguishers were in their designated locations and available for immediate use; that fire detectors and sprinklers were unobstructed, that transient material loading was within the analyzed limits; and fire doors, dampers, and penetration seals appeared to be in satisfactory condition. The inspectors also verified that minor issues identified during the inspection were entered into the licensee's CAP. Documents reviewed are listed in the Attachment to this report.

These activities constituted five quarterly fire protection inspection samples as defined in IP 71111.05-05.

b. Findings

No findings of significance were identified.

1R06 Flooding (71111.06)

a. Inspection Scope

The inspectors reviewed selected risk important plant design features and licensee procedures intended to protect the plant and its safety related equipment from internal flooding events. The inspectors reviewed flood analyses and design documents, including the UFSAR and engineering calculations, to identify licensee commitments. The specific documents reviewed are listed in the attachment. In addition, the inspectors reviewed licensee drawings to identify areas and equipment that may be affected by internal flooding caused by the failure of the fire suppression systems. The inspectors also reviewed the licensee's corrective action documents with respect to past flood-related items identified in the corrective action program to verify the adequacy of the corrective actions. The inspectors performed a walkdown of the following plant areas to assess the adequacy of watertight barriers and verify drains and sumps were clear of debris and were operable, and that the licensee complied with its commitments:

- Unit 1 and Unit 2 Switchgear Room Cable Vaults

This inspection constitutes one internal flooding sample as defined in Inspection Procedure 71111.06-05.

b. Findings

No findings of significance were identified.

1R11 Licensed Operator Requalification Program (71111.11)

.1 Resident Inspector Quarterly Review (71111.11Q)

a. Inspection Scope

On October 28, 2008, the inspectors observed a crew of licensed operators in the plant's simulator during licensed operator requalification examinations to verify that operator performance was adequate, evaluators were identifying and documenting crew performance problems and training was being conducted in accordance with licensee procedures. The inspectors evaluated the following areas:

- licensed operator performance;
- crew's clarity and formality of communications;
- ability to take timely actions in the conservative direction;
- prioritization, interpretation, and verification of annunciator alarms;
- correct use and implementation of abnormal and emergency procedures;
- control board manipulations;
- oversight and direction from supervisors; and
- ability to identify and implement appropriate Technical Specification actions.

The crew's performance in these areas was compared to pre-established operator action expectations and successful critical task completion requirements. Documents reviewed are listed in the Attachment to this report.

This inspection constituted one quarterly licensed operator requalification program sample as defined in IP 71111.11.

b. Findings

No findings of significance were identified.

1R12 Maintenance Effectiveness (71111.12)

Routine Quarterly Evaluations(71111.12Q)

a. Inspection Scope

The inspectors evaluated degraded performance issues involving the following risk significant system:

- Supplemental Diesel Generators

The inspectors reviewed events such as where ineffective equipment maintenance had resulted in valid or invalid automatic actuations of engineered safeguards systems and independently verified the licensee's actions to address system performance or condition problems in terms of the following:

- implementing appropriate work practices;
- identifying and addressing common cause failures;
- scoping of systems in accordance with 10 CFR 50.65(b) of the maintenance rule;
- characterizing system reliability issues for performance;
- charging unavailability for performance;
- trending key parameters for condition monitoring;
- ensuring 10 CFR 50.65(a)(1) or (a)(2) classification or re-classification; and
- verifying appropriate performance criteria for SSCs (Structures Systems Components)/functions classified as (a)(2) or appropriate and adequate goals and corrective actions for systems classified as (a)(1).

The inspectors assessed performance issues with respect to the reliability, availability, and condition monitoring of the system. In addition, the inspectors verified maintenance effectiveness issues were entered into the corrective action program with the appropriate significance characterization.

This inspection constitutes one quarterly maintenance effectiveness samples as defined in Inspection Procedure 71111.12-05.

b. Findings

No findings of significance were identified.

1R13 Maintenance Risk Assessments and Emergent Work Control (71111.13)

a. Inspection Scope

The inspectors reviewed the licensee's evaluation and management of plant risk for the maintenance and emergent work activities affecting risk-significant and safety-related equipment listed below to verify that the appropriate risk assessments were performed prior to removing equipment for work:

- Emergent maintenance on October 10 through 12, 2008, to replace Unit 2 east essential service water pump.
- Planned maintenance during the week of November 10, on M2 breaker in the 345 kilo volt switchyard, Unit 1 plant air compressor and surveillance testing on Unit 2 west train of residual heat removal.
- Planned maintenance on November 21 through 25, 2008, on transformer 4 in the 765 kilo volt switchyard, Unit 2 east component cooling water train and Unit 2 east centrifugal charging pump.

These activities were selected based on their potential risk significance relative to the reactor safety cornerstones. As applicable for each activity, the inspectors verified that risk assessments were performed as required by 10 CFR 50.65(a)(4) and were accurate and complete. When emergent work was performed, the inspectors verified that the plant risk was promptly reassessed and managed. The inspectors reviewed the scope of maintenance work, discussed the results of the assessment with the licensee's probabilistic risk analyst or shift technical advisor, and verified plant conditions were consistent with the risk assessment. The inspectors also reviewed TS requirements and walked down portions of redundant safety systems, when applicable, to verify risk analysis assumptions were valid and applicable requirements were met.

These maintenance risk assessments and emergent work control activities constituted three samples as defined in IP 71111.13-05.

b. Findings

No findings of significance were identified.

1R15 Operability Evaluations (71111.15)

a. Inspection Scope

The inspectors reviewed the following issues:

- AR 00826854, Unit 2 Emergency Diesel Generator Fuel Injection Pump Replacement
- AR 00839241, Identified Air / Gas Void in Piping
- AR 00841815, Unit 2 RHR Check Valve Seat Leakage
- AR 00839239, Identified Air / Gas Void in the Unit 1 North Safety Injection Train Piping
- AR 00839240, Identified Air / Gas Void in the Unit 1 South Safety Injection Train Piping
- AR 00841539, Inadequate Masonry Block HELB Barrier

The inspectors selected these potential operability issues based on the risk-significance of the associated components and systems. The inspectors evaluated the technical adequacy of the evaluations to ensure that TS operability was properly justified and the subject component or system remained available such that no unrecognized increase in risk occurred. The inspectors compared the operability and design criteria in the appropriate sections of the TS and USAR to the licensee's evaluations, to determine whether the components or systems were operable. Where compensatory measures were required to maintain operability, the inspectors determined whether the measures in place would function as intended and were properly controlled. The inspectors determined, where appropriate, compliance with bounding limitations associated with the evaluations. Additionally, the inspectors also reviewed a sampling of corrective action documents to verify that the licensee was identifying and correcting any deficiencies associated with operability evaluations. Documents reviewed are listed in the Attachment to this report.

This operability inspection constituted six samples as defined in IP 71111.15-05

b. Findings

No findings of significance were identified.

1R18 Plant Modifications (71111.18)

Temporary Plant Modifications

a. Inspection Scope

The inspectors reviewed the following temporary modification:

- U1 Reactor Coolant System Reactor Vessel Vent Assembly

The inspectors compared the temporary configuration changes and associated 10 CFR 50.59 screening and evaluation information against the design basis, the UFSAR, and the TS, as applicable, to verify that the modification did not affect the operability or availability of the affected system. The inspectors also compared the licensee's information to operating experience information to ensure that lessons learned from other utilities had been incorporated into the licensee's decision to implement the temporary modification. The inspectors, as applicable, performed field verifications to ensure that the modifications were installed as directed; the modifications operated as expected; modification testing adequately demonstrated continued system operability, availability, and reliability; and that operation of the modifications did not impact the operability of any interfacing systems. Lastly, the inspectors discussed the temporary modification with operations, engineering, and training personnel to ensure that the individuals were aware of how extended operation with the temporary modification in place could impact overall plant performance.

This inspection constituted one temporary modification sample as defined in IP 71111.18-05.

b. Findings

No findings of significance were identified.

1R19 Post-Maintenance Testing (71111.19)

a. Inspection Scope

The inspectors reviewed post-maintenance testing for the following activities to verify that procedures and test activities were adequate to ensure system operability and functional capability:

- Unit 2 east essential service water pump replacement;
- Unit 2 east centrifugal charging pump preventive maintenance;
- Unit 1 plant air compressor corrective maintenance;
- Unit 2 Train A solid state protection system logic card replacement; and
- Unit 12 east diesel driven fire pump replacement.

These activities were selected based upon the structure, system, or component's ability to impact risk. The inspectors evaluated these activities for the following (as applicable): the effect of testing on the plant had been adequately addressed; testing was adequate for the maintenance performed; acceptance criteria were clear and demonstrated operational readiness; test instrumentation was appropriate; tests were performed as written in accordance with properly reviewed and approved procedures; equipment was returned to its operational status following testing (temporary modifications or jumpers required for test performance were properly removed after test completion), and test documentation was properly evaluated. The inspectors evaluated the activities against TS, the UFSAR, 10 CFR Part 50 requirements, licensee procedures, and various NRC generic communications to ensure that the test results adequately ensured that the equipment met the licensing basis and design requirements. In addition, the inspectors reviewed corrective action documents associated with post-maintenance tests to determine whether the licensee was identifying problems and entering them in the CAP and that the problems were being corrected commensurate with their importance to safety. Documents reviewed are listed in the Attachment to this report.

This inspection constituted five post-maintenance testing sample as defined in IP 71111.19-05.

b. Findings

No findings of significance were identified.

1R20 Outage Activities (71111.20)

.1 Unit 1 Forced Outage

a. Inspection Scope

Unit 1 remained in a forced outage, which commenced on September 20, 2008, when the main turbine was manually tripped due to high vibrations and a resultant fire in the main generator. The inspectors conducted outage inspection activities, which included:

assessing the licensee's control of plant configuration and management of shutdown risk; reviewing configuration management to verify that the licensee maintained defense-in-depth with respect to shutdown risk; and, verified that systems required for decay heat removal were appropriately controlled. Outage inspection activities will be completed when Unit 1 is returned to service.

An inspection sample was not completed during this inspection period.

b. Findings

No findings of significance were identified.

1R22 Surveillance Testing (71111.22)

a. Inspection Scope

The inspectors reviewed the test results for the following activities to determine whether risk-significant systems and equipment were capable of performing their intended safety function and to verify testing was conducted in accordance with applicable procedural and TS requirements:

- Unit 2 West Centrifugal Charging Pump Surveillance Test (in-service testing)
- Unit 1 CD Diesel Generator Fast Start Surveillance Test (routine)
- Unit 2 CD 250 VDC Battery Quarterly Surveillance Test (routine)
- Reactor Coolant Pump Seal Injection Flow Resistance Surveillance Test (routine)
- Unit 2 West Residual Heat Removal Train Surveillance Test (in-service testing)
- Unit 2 Train A Reactor Protection System and Engineered Safeguards Reactor Trip Breaker and Solid State Protection System Automatic Trip/Actuation Operational Test (routine)

The inspectors observed in plant activities and reviewed procedures and associated records to determine the following:

- did preconditioning occur;
- were the effects of the testing adequately addressed by control room personnel or engineers prior to the commencement of the testing;
- were acceptance criteria clearly stated, demonstrated operational readiness, and consistent with the system design basis;
- plant equipment calibration was correct, accurate, and properly documented;
- as-left setpoints were within required ranges; and the calibration frequency were in accordance with TSs, the USAR, procedures, and applicable commitments;
- measuring and test equipment calibration was current;
- test equipment was used within the required range and accuracy; applicable prerequisites described in the test procedures were satisfied;
- test frequencies met TS requirements to demonstrate operability and reliability; tests were performed in accordance with the test procedures and other applicable procedures; jumpers and lifted leads were controlled and restored where used;
- test data and results were accurate, complete, within limits, and valid;
- test equipment was removed after testing;

- where applicable for inservice testing activities, testing was performed in accordance with the applicable version of Section XI, American Society of Mechanical Engineers code, and reference values were consistent with the system design basis;
- where applicable, test results not meeting acceptance criteria were addressed with an adequate operability evaluation or the system or component was declared inoperable;
- where applicable for safety-related instrument control surveillance tests, reference setting data were accurately incorporated in the test procedure;
- where applicable, actual conditions encountering high resistance electrical contacts were such that the intended safety function could still be accomplished;
- prior procedure changes had not provided an opportunity to identify problems encountered during the performance of the surveillance or calibration test;
- equipment was returned to a position or status required to support the performance of its safety functions; and
- all problems identified during the testing were appropriately documented and dispositioned in the CAP.

Documents reviewed are listed in the Attachment to this report.

This inspection constituted four routine surveillance testing samples and two in-service testing samples as defined in IP 71111.22, Sections -02 and -05.

b. Findings

No findings of significance were identified.

1EP4 Emergency Action Level and Emergency Plan Changes (71114.04)

a. Inspection Scope

Since the last NRC inspection of this program area, Emergency Plan Revision 26 and implementing procedure PMP-2080-EPP-101, "Emergency Classification," Revision 12 were implemented based on your determination, in accordance with 10 CFR 50.54(q), that the changes resulted in no decrease in effectiveness of the Plan, and that the revised Plan as changed continues to meet the requirements of 10 CFR 50.47(b) and Appendix E to 10 CFR Part 50. The inspectors conducted a sampling review of the Emergency Plan changes and a review of the Emergency Action Level changes to evaluate for potential decreases in effectiveness of the Plan. However, this review does not constitute formal NRC approval of the changes. Therefore, these changes remain subject to future NRC inspection in their entirety.

This Emergency Action Level and Emergency Plan changes inspection constituted one sample as defined in IP 71114.04-05.

b. Findings

No findings of significance were identified.

#### 1EP6 Drill Evaluation (71114.06)

##### a. Inspection Scope

The inspectors observed a simulator training evolution for licensed operators on October 28, 2008, which required emergency plan implementation. Licensee emergency preparedness personnel had pre-designated that the opportunities for the Shift Manager to classify the event and make required notifications would be evaluated and included in performance indicator data regarding drill and exercise performance.

The inspectors verified that the Shift Manager classified the emergency condition and completed the required notifications to state and local police authorities in an accurate and timely manner as required by the Emergency Plan implementing procedures. The inspectors also observed the post-training critique to verify that licensee evaluators appropriately identified performance deficiencies. Documents reviewed are listed in the Attachment to this report.

This emergency preparedness drill inspection constituted one sample as defined in IP 71114.06-05.

##### b. Findings

No findings of significance were identified.

#### 4OA2 Identification and Resolution of Problems (71152)

##### .1 Routine Review of items Entered Into the Corrective Action Program

##### a. Scope

As part of the various baseline inspection procedures discussed in previous sections of this report, the inspectors routinely reviewed issues during baseline inspection activities and plant status reviews to verify that they were being entered into the licensee's CAP at an appropriate threshold, that adequate attention was being given to timely corrective actions, and that adverse trends were identified and addressed. Attributes reviewed included: the complete and accurate identification of the problem; that timeliness was commensurate with the safety significance; that evaluation and disposition of performance issues, generic implications, common causes, contributing factors, root causes, extent of condition reviews, and previous occurrences reviews were proper and adequate; and that the classification, prioritization, focus, and timeliness of corrective actions were commensurate with safety and sufficient to prevent recurrence of the issue. Minor issues entered into the licensee's CAP as a result of the inspectors' observations are included in the attached List of Documents Reviewed.

These routine reviews for the identification and resolution of problems did not constitute any additional inspection samples. Instead, by procedure they were considered an integral part of the inspections performed during the quarter and documented in Section 1 of this report.

b. Findings

No findings of significance were identified.

.2 Daily Corrective Action Program Reviews

a. Scope

In order to assist with the identification of repetitive equipment failures and specific human performance issues for follow-up, the inspectors performed a daily screening of items entered into the licensee's CAP. This review was accomplished through inspection of the station's daily condition report packages.

These daily reviews were performed by procedure as part of the inspectors' daily plant status monitoring activities and, as such, did not constitute any separate inspection samples.

b. Findings

No findings of significance were identified.

.3 Semi-Annual Trend Review

a. Scope

The inspectors performed a review of the licensee's CAP and associated documents to identify trends that could indicate the existence of a more significant safety issue. The inspectors' review was focused on repetitive equipment issues, but also considered the results of daily inspector CAP item screening discussed in Section 4OA2.2 above, licensee trending efforts, and licensee human performance results. The inspectors' review nominally considered the six month period of January 2008 through June 2008 although some examples expanded beyond those dates where the scope of the trend warranted.

The review also included issues documented outside the normal CAP in departmental problem/challenges lists, system health reports, self assessment reports, and Maintenance Rule assessments. The inspectors compared and contrasted their results with the results contained in the licensee's CAP trending reports. Corrective actions associated with a sample of the issues identified in the licensee's trending reports were reviewed for adequacy.

This review constituted a single semi-annual trend inspection sample as defined in IP 71152-05.

b. Findings

No findings of significance were identified.

.4 Selected Issue Follow-Up Inspection: In-Depth Apparent Cause Evaluation

a. Scope

The inspectors selected the following action request for an in-depth review:

- In-Depth Apparent Cause Evaluation, AR 00835671, Undesired Transfer of Inventory from the Unit 2 Volume Control Tank to the Unit 2 Refueling Water Storage Tank

The inspectors discussed the evaluations and associated corrective actions with licensee personnel and verified the following attributes during their review of the above apparent cause evaluations and other related documents:

- complete and accurate identification of the problem in a timely manner commensurate with its safety significance and ease of discovery;
- consideration of the extent of condition, generic implications, common cause and previous occurrences;
- evaluation and disposition of operability/reportability issues;
- classification and prioritization of the resolution of the problem, commensurate with safety significance;
- identification of the root and contributing causes of the problem; and
- identification of corrective actions, which were appropriately focused to correct the problem.

The above constitutes completion of one in-depth problem identification and resolution sample as defined in IP 71152-05

b. Findings

No findings of significance were identified.

40A5 Other Activities

Implementation of Temporary Instruction (TI) 2515/176, "Emergency Diesel Generator Technical Specification Surveillance Requirements Regarding Endurance and Margin Testing"

a. Inspection Scope

The objective of TI 2515/176 was to gather information to assess the adequacy of nuclear power plant emergency diesel generator endurance and margin testing as prescribed in plant-specific TS. The inspectors reviewed the licensee's TS, procedures, and calculations, and interviewed licensee personnel to complete the TI. The information gathered for this TI was forwarded to the Office of Nuclear Reactor Regulation for further review and evaluation on December 17, 2008. This TI is complete at D.C. Cook Nuclear Power Plant; however, TI 2515/176 will not expire until August 31, 2009. Additional information may be required after review by the Office of Nuclear Reactor Regulation.

b. Findings

No findings of significance were identified.

4OA6 Management Meetings

.1 Exit Meeting Summary

- On January 8, 2009, the inspectors presented the inspection results to Mr. L. Weber, and other members of the licensee staff. The licensee acknowledged the issues presented. The inspectors confirmed that none of the potential report input discussed was considered proprietary.

.2 Interim Exit Meetings

- A telephone exit for TI 2515/176 was conducted with Mr. R Lingle, System Engineering Manager, and other licensee staff on December 1, 2008. The inspectors confirmed that none of the potential report input discussed was considered proprietary.
- A telephone exit for the annual review of emergency action level and emergency plan changes was conducted with the licensee's Emergency Preparedness Coordinators, Ms. C. Graffenius and Mr. D. Walton on November 13, 2008. The inspectors confirmed that none of the potential report input discussed was considered proprietary.

ATTACHMENT: SUPPLEMENTAL INFORMATION

**SUPPLEMENTAL INFORMATION**

**KEY POINTS OF CONTACT**

Licensee

S. Adkins, Regulatory Affairs/Licensing Coordinator  
L. Bush, Operations Manager  
J. Gebbie, Plant Manager  
C. Graffenius, Emergency Preparedness Coordinator  
J. Jensen, Site Support Services Vice President  
C. Hutchinson, Emergency Preparedness Manager  
C. Lane, Engineering Programs Manager  
Q. Lies, Engineering Director  
J. Newmiller, Licensing Activities Coordinator  
J. Nimtz, Licensing Activities Coordinator  
P. Schoepf, Manager Nuclear Regulatory Compliance  
D. Walton, Emergency Preparedness Coordinator  
L. Weber, Site Vice President

**LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED**

Opened and Closed

NONE

-

Discussed

NONE

## LIST OF DOCUMENTS REVIEWED

The following is a list of documents reviewed during the inspection. Inclusion on this list does not imply that the NRC inspectors reviewed the documents in their entirety, but rather, that selected sections of portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document on this list does not imply NRC acceptance of the document or any part of it, unless this is stated in the body of the inspection report.

### 1R01 Adverse Weather Protection

- 12-IHP-5040-EMP-004, Plant Winterization and De-Winterization, Revision 12, October 23, 2008
- 12-OHP-4022-001-010, Severe Weather, Revision 6, October 6, 2008
- 12-OHP-5030-057-001, Revision 18, October 28, 2008
- AR 00830909, Hurricane Shutters Continue to Fail
- AR 00839816, Degraded Heat Trace on Unit 1 PWST
- AR 07327004, Dampers for 12-OME-250-SDG2 Not Closing Properly
- AR 08109063, 1-HTDP-608-A3 Heat Trace Panel Has No Power
- AR 08312004, U1 PWST Piping Alarm After Replacement
- AR 08312047, Breaker 3 Inside the Panel Found Tripped 2 Days in a Row
- AR 08318070, Main Steam Enclosure Exhaust Fan Does Not Turn
- PMP-5055-001-001, Winterization/Summerization, Revision 7, November 10, 2008
- WR 06366195, 1-HV-MSE-D7 Will Not Shut

### 1R04 Equipment Alignment

- 1-OHP-4021-017-002, Placing In Service The Residual Heat Removal System, Revision 22, March 27, 2008
- 1-OHP-4021-008-002, Placing Emergency Core Cooling System in Standby Readiness, Revision 20, February 15, 2008
- 1-OHP-4022-017-001, Loss of RHR Cooling, Revision 18, November 30, 2007
- 1-OHP-4021-032-008CD, Operating DG1CD Subsystems, Revision 9, December 22, 2005
- 2-OHP-4021-008-002, Placing Emergency Core Cooling System in Standby Readiness, Revision 21
- CR 06102047, 2-RH-116E E-RHR Heat Exchanger Inlet Valve Leaks By About ½ gpm
- OP-2-5143-68, Flow Diagram Emerg. Core Cooling (RHR) Unit No. 2
- OP-1-5151C-50, Flow Diagram Emergency Diesel Generator "CD", Revision 50, April 12, 2002
- AR 08344052, CD FO Oil Tank Room Needs Cleaning
- AR 08151020, RCP 4 Lower Bearing Component Cooling Water Flow Low
- AR 08305013, 1-CS-441-3 Dry Acid on Cap
- AR 08305010, 1-NRV-102 Bypass Check Valve Body/Bonnet/Cap Leak
- AR 08305011, 1-SI-157-lp4 Dry Acid on Cap
- AR 08305014, 1-IMO-315 Dry Acid
- AR 08291021, Commitment 1118 and 5542 not Followed

### 1R05 Fire Protection

- 12-FPP-2270-066-001, Portable Fire Extinguisher Inspections, Revision 7, August 27, 2008
- 12-PPP-4030-066-026, Data Sheet 2, TRM Fire Door Visual Inspection, November 26 – 30
- AR 08025075, Detector Malfunctioning
- AR 08231083, Sustainability of NRC Commitments Could be Jeopardized

- AR 08331048, Door Does Not Latch
- AR 08344047, Threshold Broke Off
- AR 08347037, Mistaken Call on Operability / Functionality of Fire Door
- AR 08354038, Fire Damper in Fire Zone 17C
- AR 08354069, Unsecure Vital Area Door
- Fire Hazards Analysis, Revision 13
- Fire Pre-Plan, Revision 4
- NFPA 80-1970, Standard for Fire Doors and Windows
- WR 06366136, Multiple Invalid Fire Alarms
- WR 06366941, 1-VSX-1501 is Inoperable

#### 1R06 Flooding

- 01-OHP-4025-LS-3, Steam Generator 2/3 Level Control, Revision 3
- 01-OHP-4025-LS-4, Steam Generator 1/4 Level Control, Revision 3
- 12-PPP-4030-066-026, Technical Requirements Manual Fire Door Inspection, Revision 2
- 1-OHP-4025-001-001, Emergency Remote Shutdown, Revision 6
- CR 06061038, Tracking AR for the Recommended Risk-Informed Design Change
- Fire Hazards Analysis, Revision 13
- Fire Protection Program Manual, Revision 10
- Internal Flooding Analysis Notebook, D.C. Cook Unit 1 & 2, Revision 0
- OP-12-5152D-12, Flow Diagram Fire Protection-Water Auxiliary & Containment Buildings Unit 1 & 2, Revision 12
- PRA-FLOOD-001, Evaluation of Rupture of Fire Protection (Water) Piping in Fire Zone 55, Revision 0
- PRA-FLOOD-002, Internal Flooding Impact Power Distribution, Revision 0
- PRA-FLOOD-008, Flood Sources and the Associated Flooding Mechanisms, Revision 0
- Regulatory Guide 1.102, Flood Protection For Nuclear Power Plant, Revision 1
- SD-061206-001, Flooding Evaluation Report, Revision 0
- Updated Final Safety Analysis Report, Revision 22

#### 1R11 Licensed Operator Regualification Program

- Crew Periodic Simulator Evaluation, Shift E, October 28, 2008
- RQ-E-3305B, Cycle 3305 As-found Simulator Evaluation B, Revision 0

#### 1R12 Maintenance Effectiveness

- 12-EHP-5035-MRP-001, Maintenance Rule Program Administration, Revision 17
- AR 00806414, Supplemental DG Motorized Disconnect Switch Opened
- AR 00813397, Both SDGs Tripped Off on Over Voltage
- AR 00813552, SDG 2 Output Breaker 12-52-G2 Tripped Open
- AR 00826051, SDG 2 Output Breaker Tripped Open and Then Diesel Tripped
- AR 00829230, Loss of 69kV Power Momentarily
- AR 00830720, 69 KV EP Lost Due To Being Tripped Off By SDGs
- AR 08121055, Unexpected Loss of 69 KV Emergency Power
- AR 08158047, Past Operability Evaluation Conclusion Basis Not Sufficient
- Maintenance Rule Scoping Document, Supplemental Diesel Generators , Revision 0
- PMI-5035, Maintenance Rule Program, Revision 13
- Two-year Unavailability Report for the Supplemental Diesel Generator System from December 9, 2006 to July 1, 2008

### 1R13 Maintenance Risk Assessments and Emergent Work Control

- Control room logs, October 10 -12, November 10 -14, November 22 - 25
- D. C. Cook Nuclear Plant PRA Study, Transformer 4 Outage Risk Assessment, PRA-Study-056, November 2008
- PMP-2291-OLR-001, On-Line Risk Management, Unit 2 Part 1 Configuration Risk Assessment, October 10 -12, November 10 -14, November 22 - 25
- Schedule of daily work activities, October 10 -12, November 10 -14, November 22 - 25

### 1R15 Operability Evaluations

- 12-4075-HELB-1, Screen and Pump House Units 1 and 2, Revision 1, February 1, 2001
- 12-QHP-5050-NDE-025, Ultrasonic Examination for Identifying Sediment and Air/Gas Voids in Piping Systems, Revision 2, September 26, 2008
- 2-OHP-4030-208-008R, ECCS Check Valve Test, Revision 9, October 16, 2007
- 2-OHP-4030-217-050E, East RHR Train Operability Test Modes 1-4, Revision 5, October 2, 2008
- 2-OHP-4030-217-050W, West RHR Train Operability Test Modes 1-4, Revision 9, August 14, 2008
- AR 00813401, Calculation TH-95-01 Does Not Model Gap at Column Line A
- AR 00831780, Incomplete Non Intrusive Test of 2-FW-135
- AR 00840100, Loss of Configuration Unit 2 CD EDG Fuel Injection Pumps
- AR 00841702, 2-RH-164E Failed Full Closure Test
- AR 08080063, Non-conservative SI Pump Curve Modeled in W Safety Analysis
- AR 08254019, Corrosion discovered on 2-Batt-CD Rack
- AR 08325073, Check Valve 1-RH-164E Touching Insulation of Adjacent Pipe.
- AR 08338069, Unit 2 LEFM NCAL
- DIT-B-03234-00, Unit 2 Engineering Change-47742 RHR Crosstie Modification Flow Criteria, September 18, 2007
- EE-2006-0021, Emergency Diesel Generator Fuel Injection System Engineering Evaluation, Revision 0, March 16, 2006
- NRC Generic Letter 2008-01, Managing Gas Accumulation in Emergency Core Cooling, Decay Heat Removal, and Containment Spray Systems, January 11, 2008
- OP-1-5142, 1-SI-12 Elevation Flow Diagram, Revision 15, July 29, 2006
- PMP-5040-IEE-001, Item Equivalency Evaluations, Revision 8, April 27, 2007
- WO 55328699, 1-SI-120S Identified Air/Gas Void in Piping
- WO 55328700, 1-SI-120N Identified Air/Gas Void in Piping

### 1R18 Plant Modification

- 1-OHP-4021-001-004, Plant Cooldown From Hot Standby to Cold Shutdown, Revision 55, October 15, 2008
- AR 00840401, Cat ID#0497001632 Set Up in Stock But Has No Parts
- AR 08305008, Reactor Coolant Pump 12 Large Pile of Dry Boric Acid in Bowl, Some Discolored
- AR 08305009, Reactor Coolant Pump 13 Large Pile of Dry Boric Acid in Bowl, Some Discolored
- Work Order Package 55329769, October 14, 2008

## 1R19 Post Maintenance Testing

- 12-IHP-4030-066-001, Fire Pump Performance and Starting Sequence Tests, October 4, 2008
- 12-IHP-5021-EMP-080, Eaton/Cutler-Hammer 4KV Circuit Breaker Maintenance
- 12-OHP-4030-066-121FD, Diesel Fire Pump Operability Test, October 4, 2008
- 1-OHP-4021-064-001, Operation of Plant and Control Air Systems, November 12, 2008
- 2-IHP-4030-STP-510, Train A RPS and ESF Reactor Trip Breaker and SSPS Automatic Trip/Actuation Logic, October 10, 2008
- 2-OHP-4030-203-052E, East Centrifugal Charging Pump Operability Test, Revision 6, November 25, 2008
- 2-OHP-4030-219-022E, East Essential Service Water System Test, October 11, 2008
- AR 08284015, Work Packages Not Ready for 2-PP-7E WCA Dept Clock Reset
- AR 08287022, Delays encountered during emergent ESW Pump/Motor Work
- AR 08324007, Ensure Activity for 1PP50W rebuild during U1C23
- AR 08325053, 2E-CCP Outboard Mechanical Seal Leakage
- AR 08350122, 1-PRV-1, Inadequate Documentation / Missed PMT
- Work Order 55328169, 12-PP-145E, East Diesel Driven Fire Pump, October 2, 2008
- Work Order 55329252, 2-1 Steam Generator Water Level Hi-Hi Alarm, October 10, 2008
- Work Order 55331356, Plant Air Compressor OME-41 Inlet Control PRV-1 Actuator, November 12, 2008
- WR 06365697, Unit 2 East ESW Pump Has HI Vibrations

## 1R22 Surveillance Testing

- 12-IHP-4030-082-002, AB, CD and N Train Battery Quarterly Surveillance and Maintenance, Revision 16, November 6, 2008
- 1-2-UNC-090-CALC1, RCP Seal Water Injection Flow, February 5, 2000
- 1-OHP-4030-132-127CD, CD Diesel Generator Operability Test (Train A), Revision 5, October 28, 2008
- 2-IHP-4030-STP-510, Train 'A' RPS and ESF Reactor Trip Breaker and SSPS Automatic Trip/Actuation Logic Operational Test, Revision 14, Performed October 7, 2008
- 2-OHP-4030-203-052L, Controlled Leakage Verification Test, Revision 5, November 12, 2008
- 2-OHP-4030-203-052W, West Centrifugal Charging Pump Operability Test, Revision 8, October 16, 2008
- 2-OHP-4030-217-050W, West Residual Heat Removal Train Operability Test Modes 1-4, Revision 9, November 13, 2008
- AR 00045167, 01/02 OHP 4030.STP.052E & W Needs An Enhancement to Delete
- AR 00049689, 2 OHP 4030.STP.052W Attachment 5 Requires That The CCP Be Declared Inoperable
- AR 00831627, Out of Tolerance Flow Instrument Impacts RHR Pump Test
- AR 00831627, Out-of-Tolerance Flow Instrument Impacts RHR Pump Test
- AR 00831630, Additional Cells Weeping Electrolyte
- AR 00831780, Incomplete Non Intrusive Test of 2-FW-135
- AR 00833622, The Inservice Test Program Plan Needs to be Revised
- AR 01082061, Operations' Surveillances for Pump Test That I&C Has Gauge Support Need Upgraded
- AR 08305060, Cell #106 is Leaking Around the Seal
- AR 08308024, Determine Extent of Leaking Pipe
- AR 08311047, 2-Batt-CD-Cell #27
- OP-2-5128A-56, Flow Diagram Reactor Coolant, August 21, 2008
- OP-2-5143-68, Flow Diagram Emergency Core Cooling (RHR) Unit 2

#### 1EP4 Emergency Action Level and Emergency Plan Changes

- D.C. Cook Nuclear Power Plant Emergency Plan, Revisions 25 and 26
- PMP-2080-101, Emergency Classification, Revisions 11 and 12
- PMP-2080-EPA-008, 50.54(q) Program Evaluation and Effectiveness Reviews, April 14, 2008

#### 1EP6 Drill Evaluation

- EMD-32a, Michigan State Police, Nuclear Plant Event Notification, October 28, 2008, Drill
- PMP-2080-EPP-100, Emergency Response, Revision 13
- PMP-2080-EPP-101, Emergency Classification, Revision 12
- TRP-2070-TAP-400-OPS, Data Sheet 13, Emergency Plan Performance Indicator, October 28, 2008

#### 4OA2 Problem Identification and Resolution

- 12-IHP-5030-EMP-001, Limitorque Valve Operator Preventive Maintenance, Revision 12, July 28, 2008
- 1st Quarter 2008 Trend Report
- 2-OHP-4030-208-053A, Emergency Core Cooling System Valve Operability Test – Train A, Revision 17
- AR 00108552, North Control Room Air Conditioning Unit Air Handling Fan
- AR 00817267, Potential Adverse Trend in Reason Code C1J
- AR 00822799, Potential Adverse Trends With Closure of Actions
- AR 00825987, U2 North Control Room Air Conditioning Fail to Start
- AR 00826118, 1-ECR-16 Open Stroke Greater Than Limit is Inoperable
- AR 00827307, Trend Analysis Equipment Failures Leading to Unplanned Limiting Condition for Operations
- AR 00827616, Red Performance Indicator for Unplanned Tech Spec Action Statement Entries
- AR 00828199, Upper Control Limit Met for Reason Code D6g1
- AR 00828584, Develop ODM for Sand/Silt Issues Facing the Plant
- AR 00832097, Adverse Trend of Foreign Material Degrading Equipment Reliability
- AR 00835671, In-Depth Apparent Cause Evaluation For An Undesired Transfer of Inventory
- AR 00835962, Reason Code A2f Exceeded Upper Control Limits
- AR 00836501, 2-IMO-362 Was Not Planned Correctly
- AR 08304054, Action Not Created From Corrective Action Evaluation
- Reason Code (Event Code) Search, 2nd Quarter 2008
- OP-2-5129-49, Flow Diagram CVCS-Reactor Letdown and Charging Unit 2, October 17, 2006
- OP-2-5142-50, Flow Diagram Emergency Core Cooling (SIS), August 9, 2006

#### 4OA5 Other Activities

- 1-OHP-4030-132-027AB; AB Diesel Generator Operability Test (Train B), Revision 5
- 1-OHP-4030-132-027CD; CD Diesel Generator Operability Test (Train A), Revision 3
- 2-OHP-4030-232-027AB; AB Diesel Generator Operability Test (Train B), Revision 4
- 2-OHP-4030-232-027CD; CD Diesel Generator Operability Test (Train A), Revision 3
- Calculation 1-E-N-ELCP-4KV-001-LOAD; Unit 1 Equipment Loading Analysis, Revision 0
- Calculation 2-E-N-ELCP-4KV-001-LOAD; Unit 2 Equipment Loading Analysis, Revision 0

## LIST OF ACRONYMS USED

ADAMS	Agency Documents Access and Management System
AR	Action Request
CAP	Corrective Action Program
CFR	Code of Federal Regulations
IMC	Inspection Manual Chapter
LER	Licensee Event Report
NRC	U.S. Nuclear Regulatory Commission
PARS	Publicly Available Records
SSC	Structure System Component
TI	Temporary Instruction
TS	Technical Specification
UFSAR	Updated Final Safety Analysis Report
USAR	Updated Safety Analysis Report